

DE LA RECHERCHE À L'INDUSTRIE
cea den

GENIORS
GEN IV Integrated Oxide fuels recycling strategies

**STAKEHOLDERS EVENT
& TOPICAL DAY**

PARTITIONING MEETS TRANSMUTATION

Partitioning for the 4th generation

A review of MA partitioning processes developed in Europe and in the world

M. Miguirditchian

*Research Department on Mining & Fuel Recycling
Processes*

CEA Marcoule, Nuclear Energy Division



Tackle the issues of MA partitioning

Composition for 1 Phenix assembly after irradiation (Cooling time 1y at 85GWd/t)

- U : 40 kg (76%)
- Pu : 8,1 kg (15,3%)
- MA (Np, Am, Cm) : 294 g (0,55%)
- FP : 4,35 kg (8,2%)

➤ SNF : Multi-element inventory

■ Recycle MA without :

- Neutronic poisons (Ln : Sm, Eu...) → impact on transmutation performances
- β - γ FPs → impact of dose rate on fabrication processes

➤ Close physico-chemical properties between elements

- MA/FP competition
- An(III)/Ln(III) separation

➤ High acidity of the solution

- M^{n+}/H^+ competition

➤ Solvent radiolysis and hydrolysis

- Chemical degradation of extracting molecules

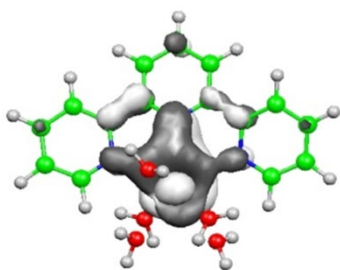
How recycling minor actinides ?



*A few hundreds
of new molecules*

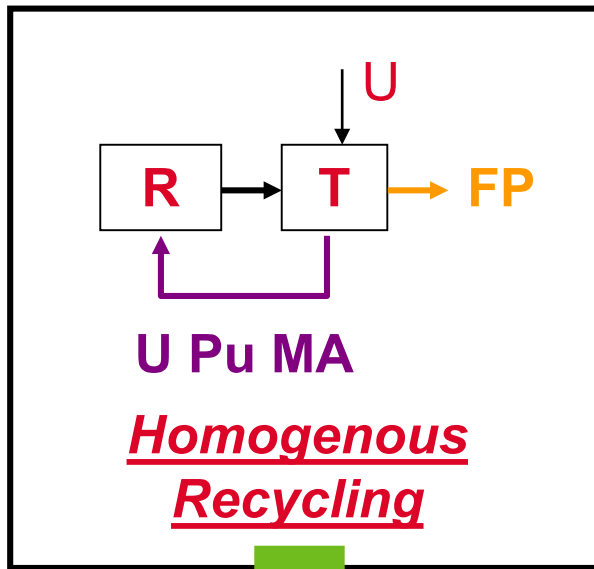


Scale : 1/100 à 1/1000



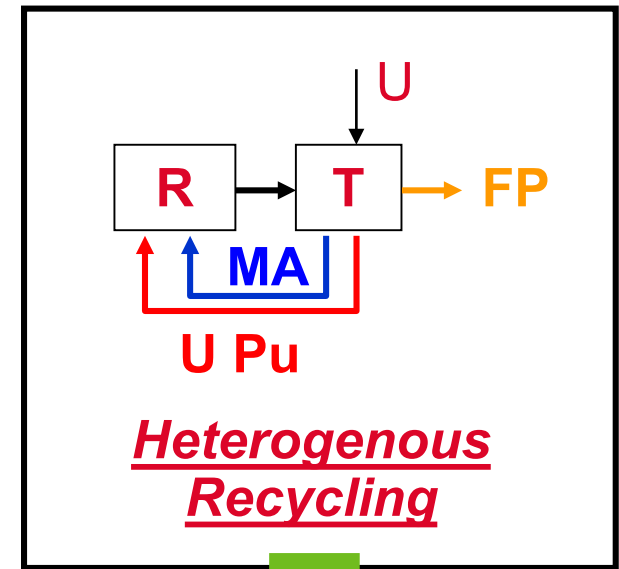
- SX as a reference option in consistency with the current industrial experience of La Hague
 - High separation yields
 - Low amount of secondary waste
 - Very significant feedback: > 25 000 t_{HM} LWR SNF reprocessed in La Hague plants
- Methodology applied for MA partitioning
 - Develop new SX processes based on new extracting or complexing molecules
 - A 3 steps methodology applied in wide cooperation framework (several EU projects ... ACSEPT... SACSESS and now GENIORS!)
 - Explorative R&D and in-depts understanding of actual mechanisms
 - Batch lab experiments and process design
 - Demonstration experiments on actual SNF (5-10 kg)
 - In ATALANTE (CEA) or JRC (EU) or NNL

Different partitioning options for the future



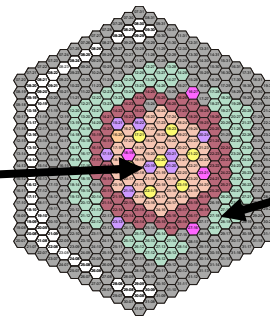
→ grouped recycling
GANEX processes

MA diluted ~2% in standard fuel in the whole core of GEN IV reactor

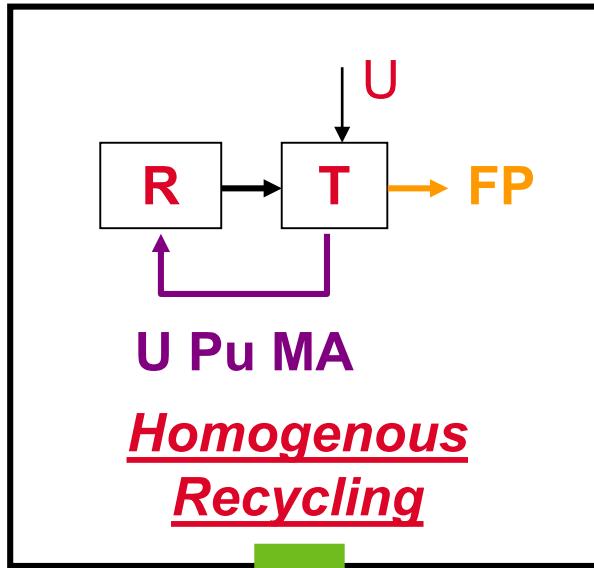


→ Enhanced partitioning
Advanced PUREX + DIAMEX-SANEX (or i-SANEX)

Moderated core target or blanket in periphery of the core with MA content up to 20% (MABB concept)

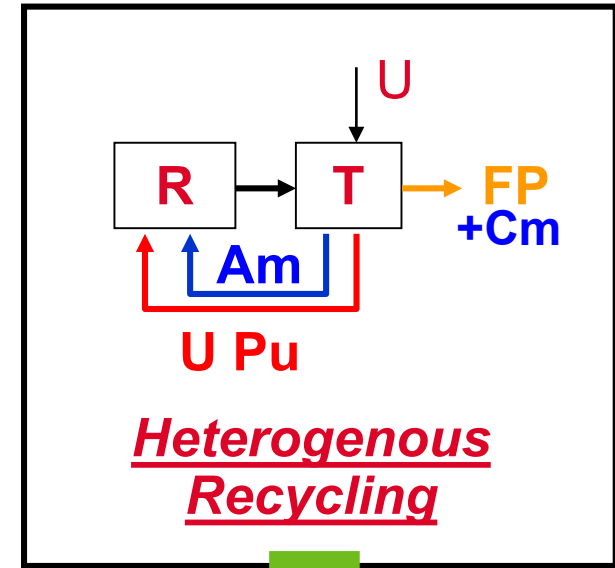


Different partitioning options for the future



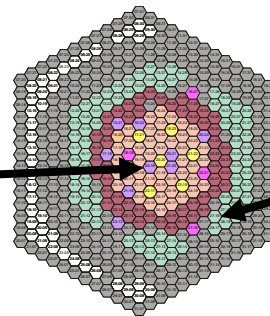
→ grouped recycling
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MA diluted ~2% in standard fuel in the whole core of GEN IV reactor



→ Enhanced partitioning
Advanced PUREX
+
EXAm

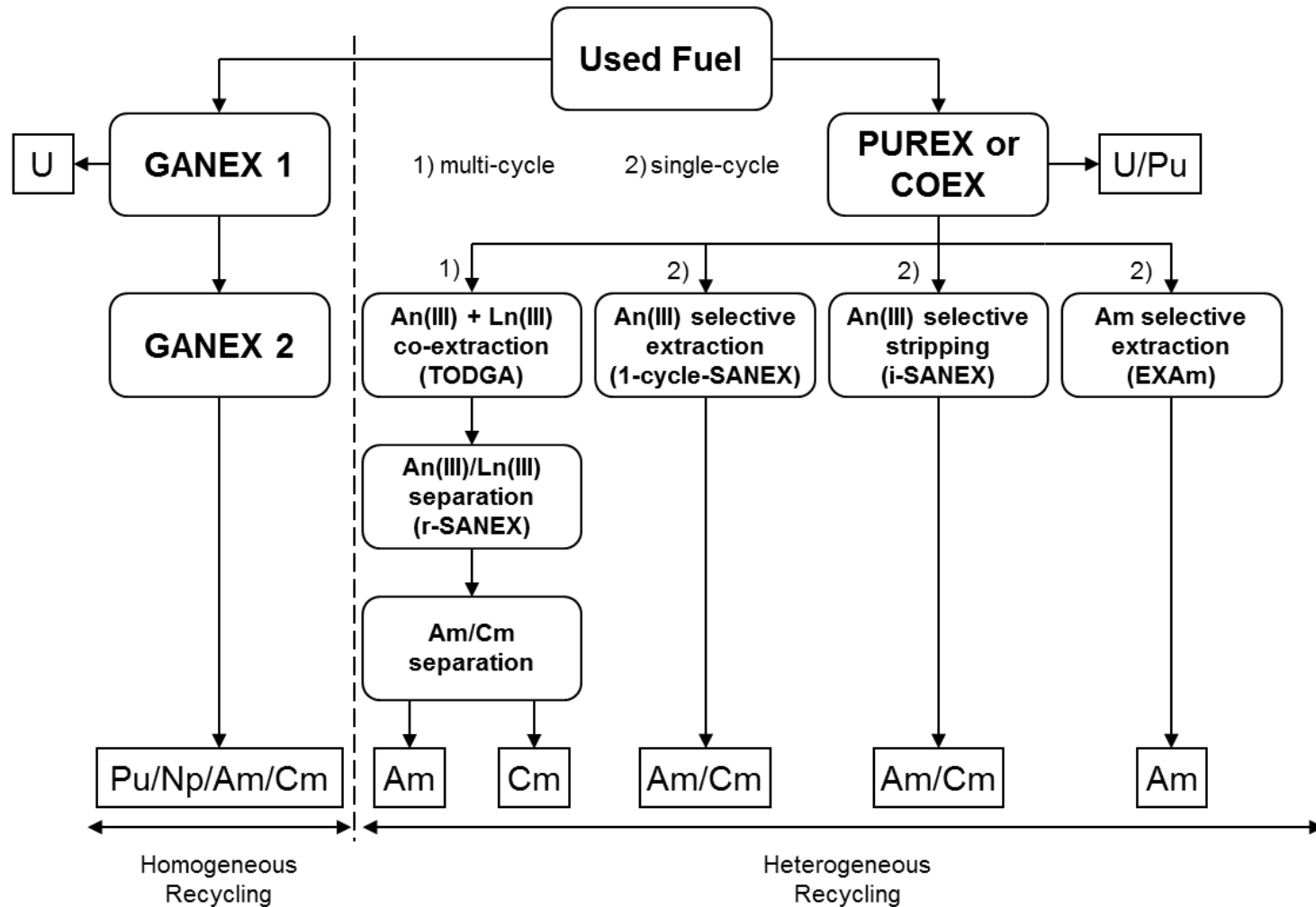
Moderated core target or blanket in periphery of the core with MA content up to 20% (MABB concept)



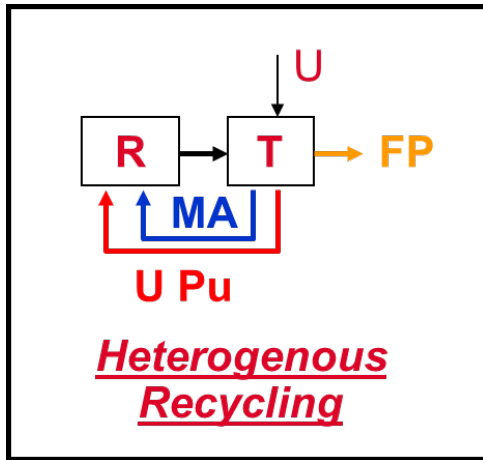
www.sacsess.eu



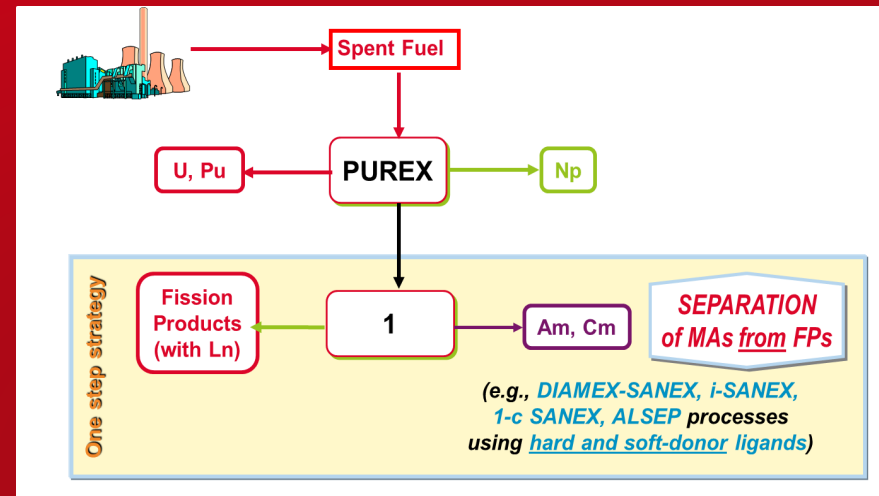
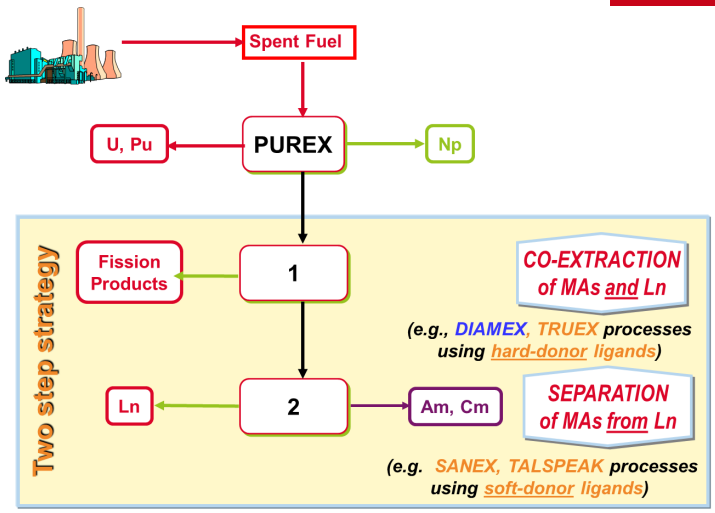
European hydrometallurgy partitioning strategy for actinide recycling



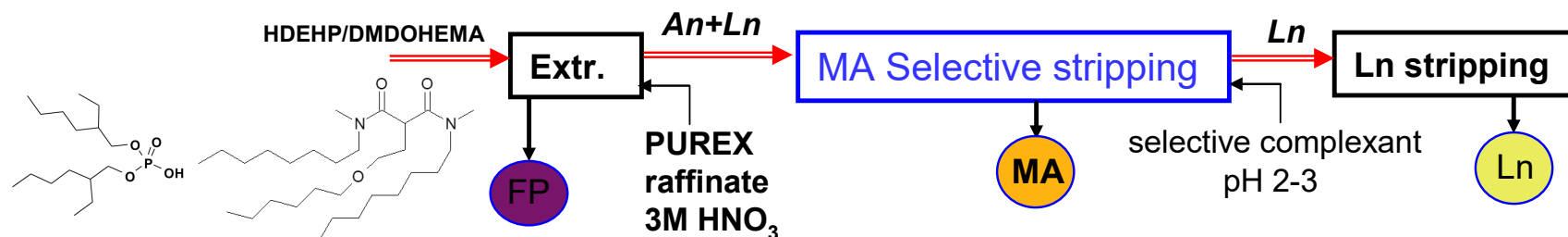
G. Modolo, A. Geist, M. Miguiditchian, « Minor actinide separations in the reprocessing of different fuels : recent advances in Europe »
Reprocessing and Recycling of Spent Nuclear Fuel - Published by Robin Taylor



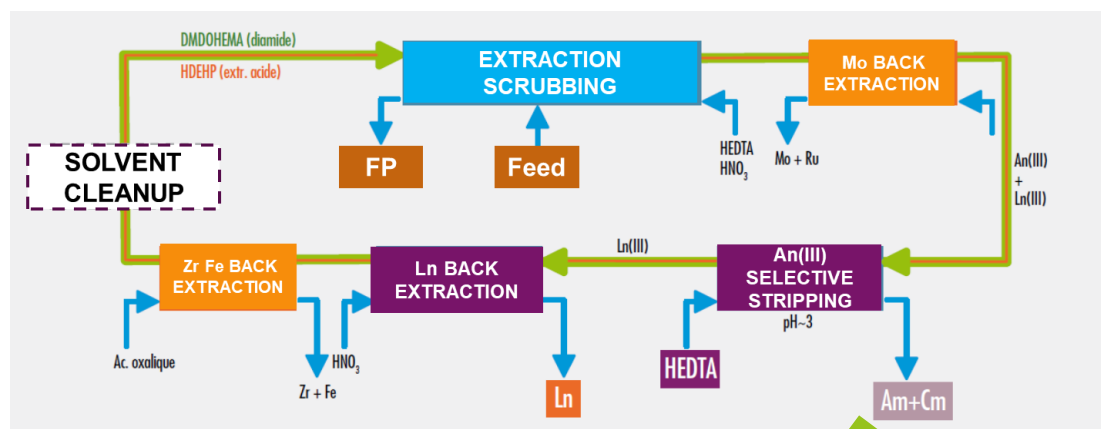
MA separation



➤ Process based on actinide separation by selective actinide stripping



- Use of mixture of HDEHP and DMDOHEMA for Am+Cm+Ln extraction
- Specific separation steps to take Mo Ru Zr Fe extraction into account
- Selective An stripping using HEDTA and citric acid at pH 3
- Demonstrated on genuine PUREX raffinates in 2000 and 2005 in Atalante facility



Am recovery > 99.9 %, Cm ~ 99.7 %
DF MA/Ln > 800

➤ Process developed by CEA, based on the « japanese » TODGA

- Use of a mixture of TODGA and TBP for An+Ln co-extraction
- Selective An stripping using DTPA and malonic acid at pH 2 with a salting out agent (NaNO_3) to maintain Ln in organic phase

😊 — No need of a 2nd cationic exchanger (HDEHP)

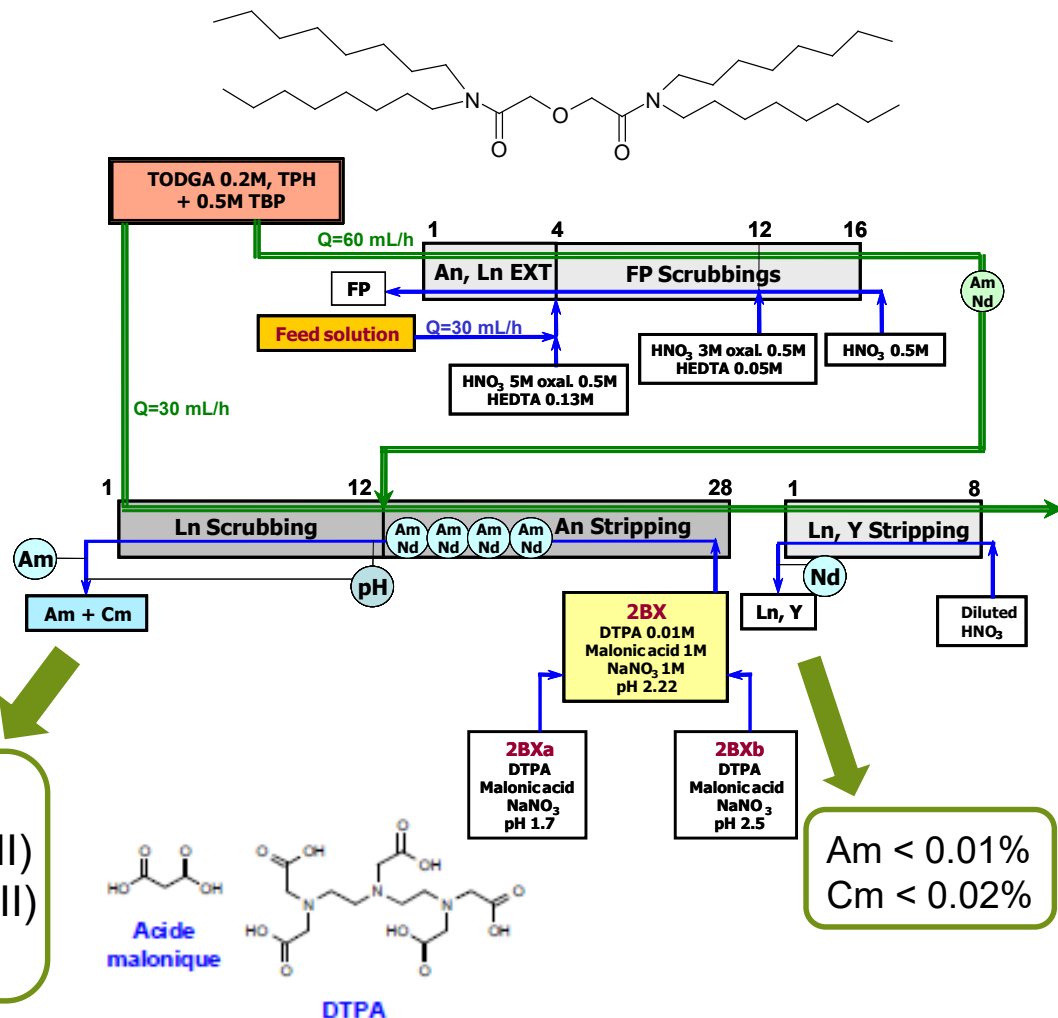
😞 — Strong sensitivity to pH

- Hot test performed in 2009 on genuine PUREX raffinate in ATALANTE in the frame of the ACSEPT project

Product:

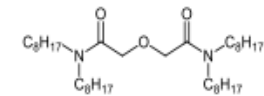
> 99.8 % Am(III)
> 99.9 % Cm(III)
< 2 % mass Ln

G. Modolo, A. Geist, M. Miguiditchian,
Reprocessing and Recycling of Spent
Nuclear Fuel - Published by Robin Taylor

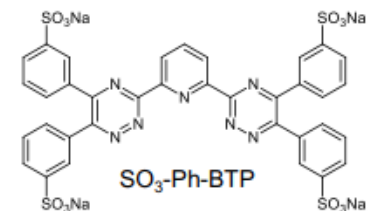


➤ Process developed by Jülich and KIT in the frame of ACSEPT

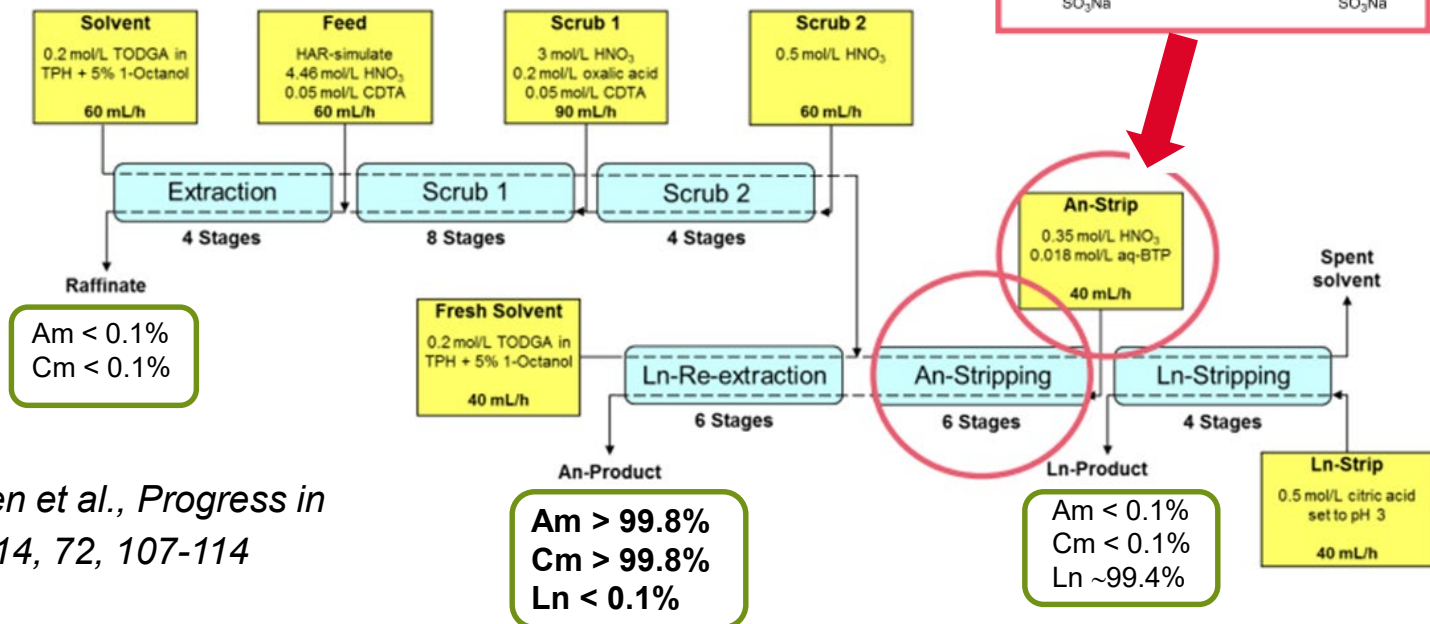
- Mixture of TODGA + 5% octanol in TPH for An+Ln co-extraction
- Selective An stripping using hydro SO₃-Ph-BTP at 0.35M HNO₃
 - ☺ — Stripping at high acidity, no need to control the pH
 - ☹ — Stability and potential downstream effect of SO₃-Ph-BTP, no CHON
- Spiked test performed in 2012 at Jülich on a surrogate PUREX raffinate



TODGA



SO₃-Ph-BTP

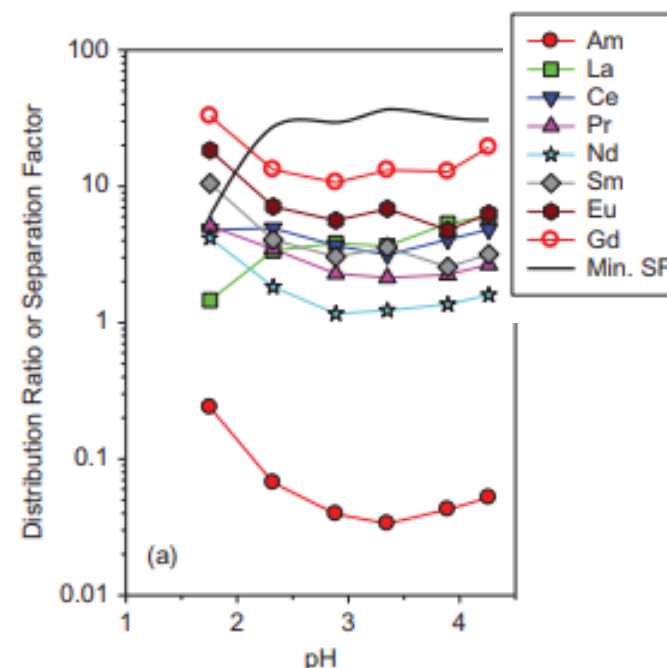
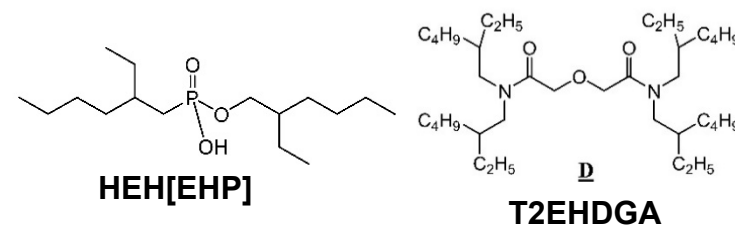


G. Modolo, A. Wilden et al., *Progress in Nuclear Energy*, 2014, 72, 107-114

➤ Process developed by US DOE based on a reverse-TALSPEAK process (similar to DIAMEX-SANEX process)

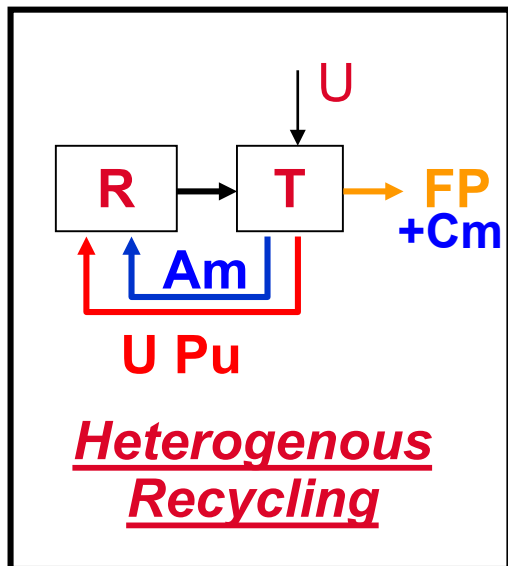
- Mixture of T2EHDGA and HEH[EHP] for An+Ln co-extraction
- Selective An stripping using DTPA and citric acid at pH 2-3
 - $SF_{Ln/Am} \sim 30$
- Spiked test performed in 2017 at ANL and planned at Jülich in centrifugal contactors on a surrogate PUREX raffinate in 2019
- Am/Cm separation studied at DOE by Am(III) oxidation and selective extraction of Am(VI)
 - Oxidation of Am(III) by sodium bismuthate
 - Selective extraction by DAAP or a monoamide

B. Mincher et al., SX-IX, 2014, 32, 153-166

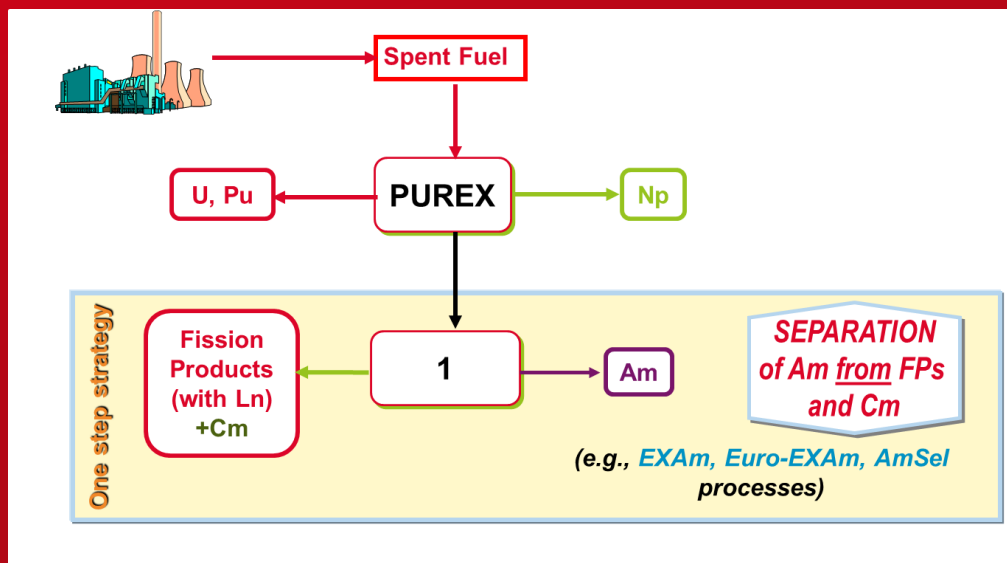


G. Lumetta et al., SX-IX, 2014, 32, 333-347

A. Guelis, I&EC, 2014, 53, 1624-1631



Am separation



- Selective Am(III) and light Ln extraction by HDEHP+DMDOHEMA solvent from 5M HNO₃

- Addition of TEDGA to increase Am/Cm selectivity from 1.6 to 2.5

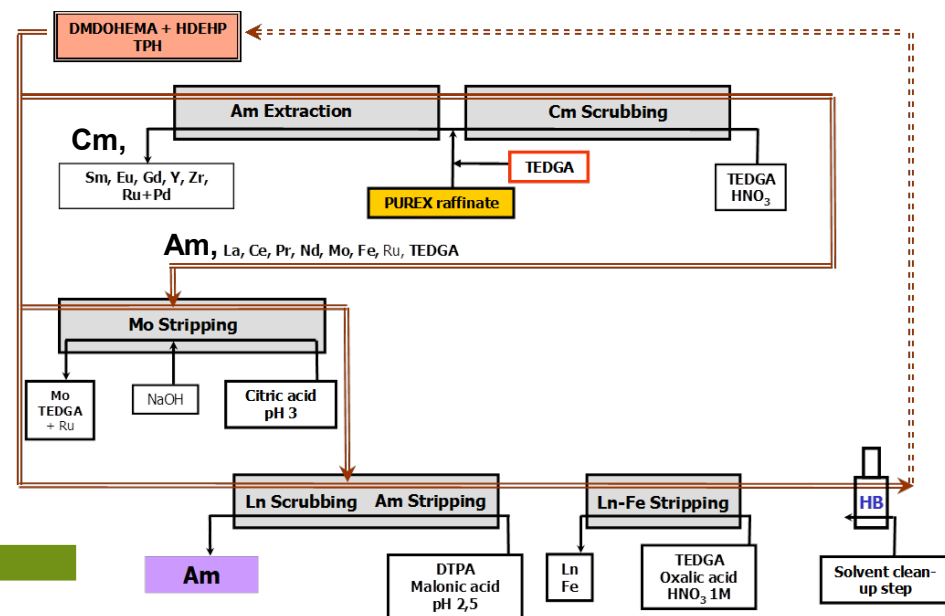
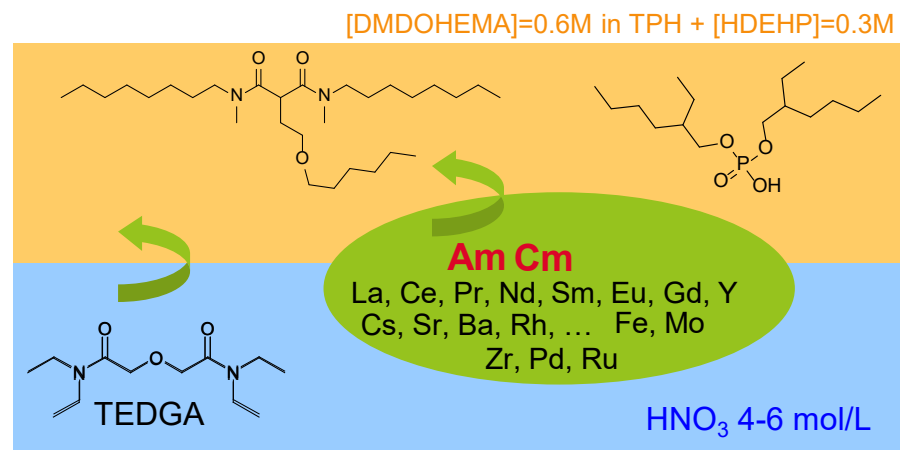
- Selective Am stripping by DTPA + malonic acid at pH 2.5
- Validated in 2010 after a hot test performed in mixer-settlers on a genuine PUREX raffinate in ATALANTE facility

- Adaptation of the EXAm process to treat a PUREX HAC (x3)

- Hot test performed in 2015 allowed to recover 2.4 g of Am (DF Am/Cm ~ 50)

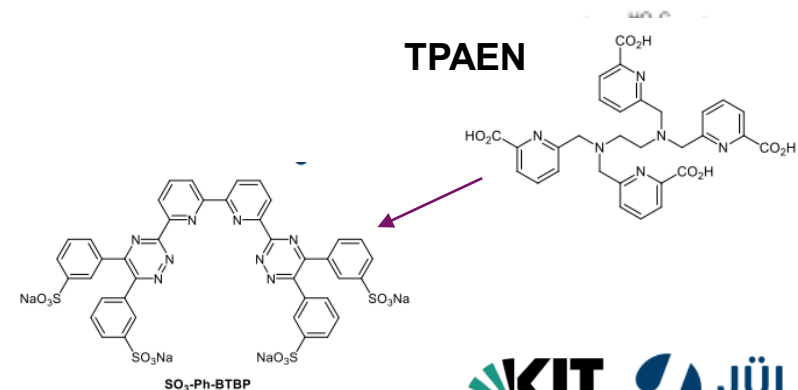
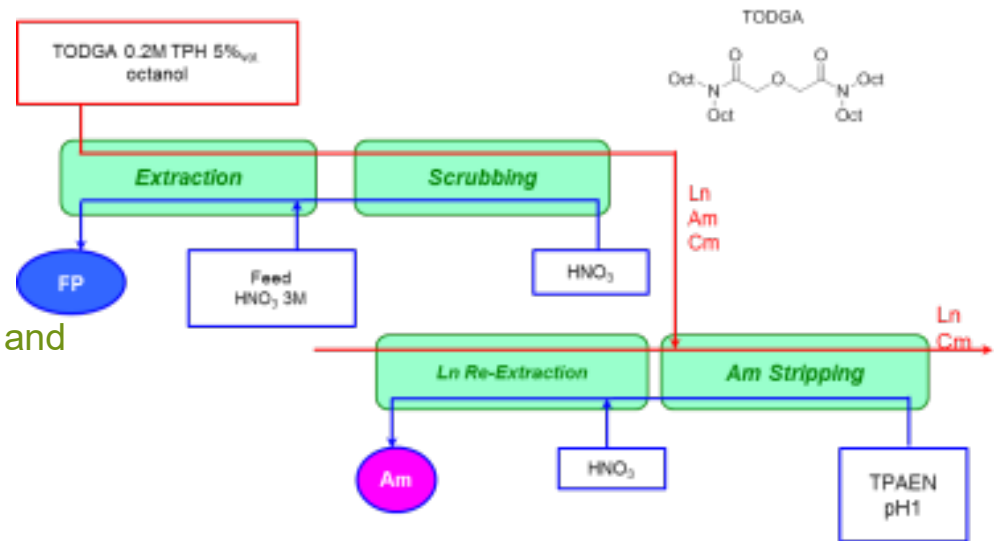
V. Vanel et al, Procedia Chemistry, 2016

~ 98.5 % Am(III)
DF Am/Cm ~ 500
DF Am/Nd ~ 340



➤ Adaptation of i-SANEX for Am selective extraction

- Mixture of TODGA + 5% octanol in TPH for An+Ln co-extraction
- Selective Am stripping using TPAEN at pH 1
 - Good Am/Cm selectivity ($SF \sim 4$)
 - No need of a 2nd cationic exchanger and no need of TEDGA nor pH buffer
 - Difficult Am/Light Ln separation
 - Low solubility of TPAEN aq. phase
- Spiked test performed in 2015 in Jülich on a surrogate PUREX raffinate
 - Solubilities issues during the spiked test
- Alternative solution studied in GENIORS with SO_3 -Ph-BTBP



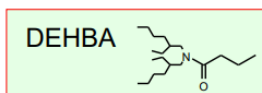
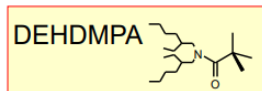
AmSel Process



- Medium Am/Cm selectivity ($SF \sim 2.5$)
- Good Am/Light Ln selectivity
- High solubility
- No CHON compound

➤ Multi-step approach to separate Am, developed at JAEA (Japan)

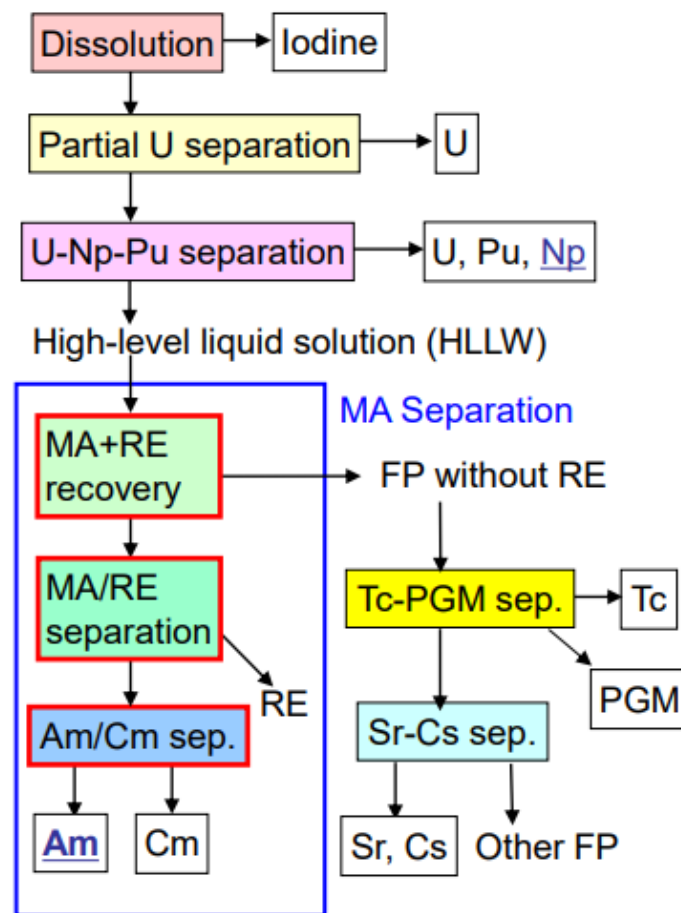
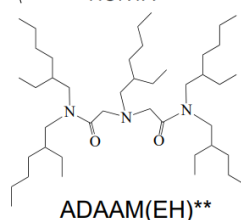
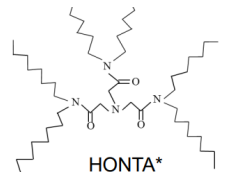
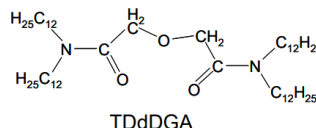
- MA+RE and MA/RE separation steps tested on genuine raffinate
- Am/Cm separation tested on a spiked solution



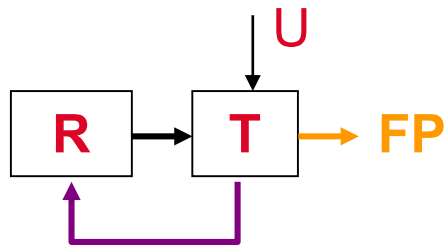
Am, Cm, Ln > 99,9%

$SF_{Am/Eu} \sim 6$
Am ~ 94.5%, Cm ~ 78%
La ~ 0.1% Nd ~ 16%

$SF_{Am/Cm} \sim 5.5$
 $SF_{Am/Cm} \sim 41$ with TEDGA
Am > 99% with Cm ~ 9.6%



H. Suzuki, T. Matsumura, et al., J. Nucl. Sci. Technol., in press.



U Pu MA

Homogenous
Recycling

MA group separation

2-step GANEX concept :

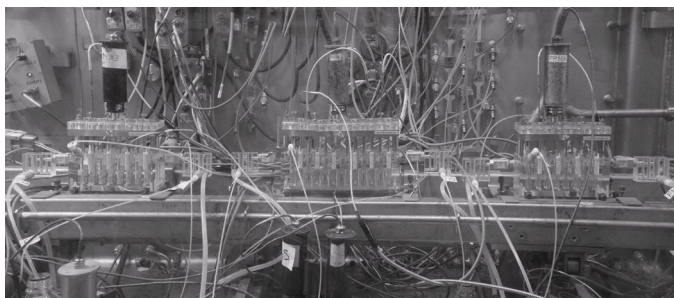
- Selective U separation by the monoamide DEHiBA (GANEX 1)
- Transuranium group separation

- Use of a mixture of HDEHP and DMDOHEMA to extraction TRU and Ln

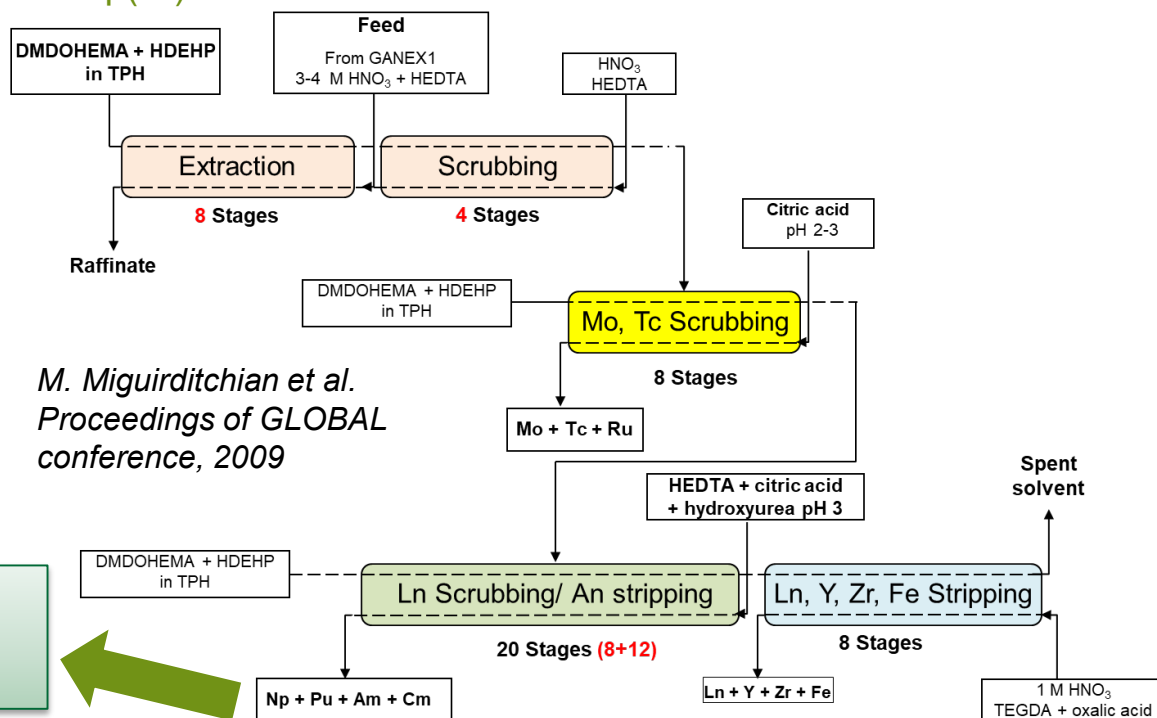
- Increase of [HDEHP] to increase loading capacity (due to high Pu content)
- Increase of [HNO₃] to extract Np(VI) along with Pu, Am, Cm

■ Addition of hydroxyurea to reduce Np(VI)

- Hot test performed in 2008 on a genuine PUREX raffinate in 48 mixer-settlers

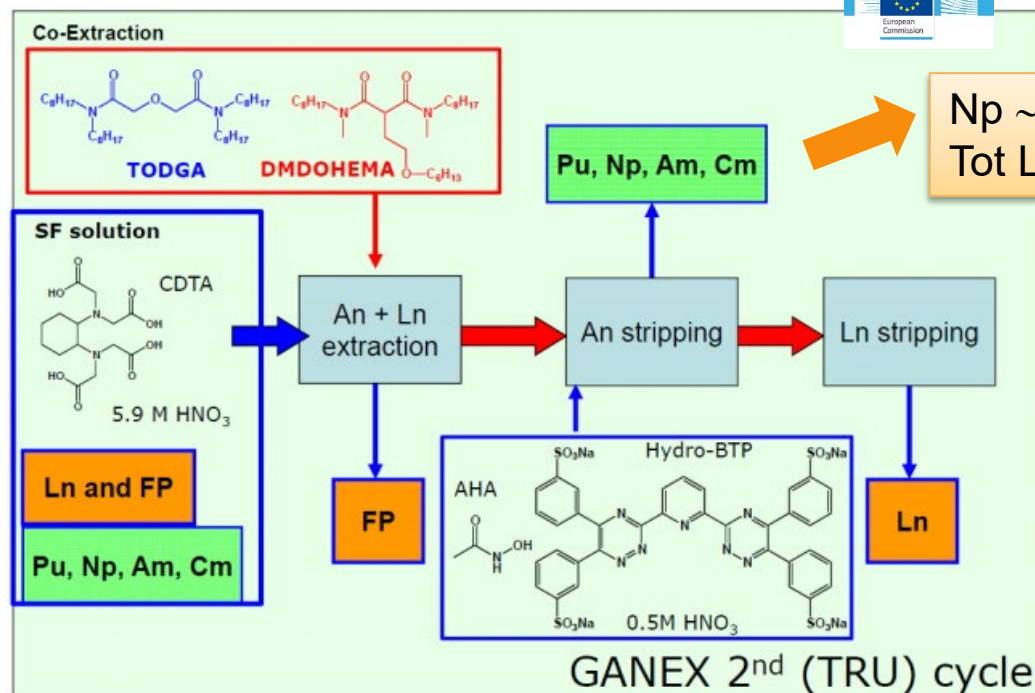


99.8% < Actinide recovery < 99.9%
DF An/FP ~ 20-40



➤ Development of group separation process based on TODGA solvent

- Use of a mixture of TODGA and DMDOHEMA to avoid Pu precipitates
 - Possible to extract high concentration of Pu along with Np, Am, Cm and Ln
- Actinide/Lanthanide separation by selective An stripping with AHA and SO₃-Ph-BTP
- Spiked tests at NNL and a hot test performed on a genuine GANEX 1 raffinate in centrifugal contactors in 2012 at JRC

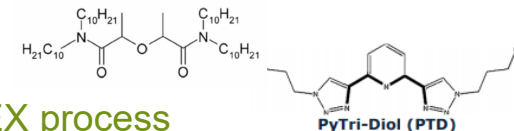


M. Carott et al. Hydrometallurgy, 2017
R. Taylor et al, Procedia Chemistry, 2016

Np ~ 99.91%, Pu ~ 99.85%, Am ~ 99.9%
Tot Ln ~ 0.06%

- Alternative solution studied in GENIORS

- Substitution of TODGA+DMDOHEMA by MeTDDGA and SO₃-Ph-BTP by PyTriDiol



- CHALMEX process
- CyMe₄BTBP + TBP in FS-13

- Efficient partitioning processes of MA have been developed and qualified
 - DIAMEX-SANEX, 1c-SANEX, i-SANEX, ALSEP, EXAm,... for heterogenous mode
 - GANEX processes for homogeneous mode
- Priority is now put on Pu multi-recycling and we need a strong international collaboration to keep developing MA partitioning processes
 - Improvement of Euro-EXAm
 - Improvement of i-SANEX and EURO-GANEX with promising mTDDGA and PyTriDiol molecules
 - Study the impact of these new partitioning processes on the conversion step
 - Impact of PTD, AHA, SO_3 -Ph-BTBP on actinide conversion
- Study the impact of a lower FP decontamination on the MA transmutation performances to develop more compact and cost-effective processes
 - Find a compromise by better linking P&T studies



Acknowledgments

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R. Malmbeck (JRC)
T. Todd (INL)
G. Lumetta (PNNL)
T. Matsumara (JAEA)

Thank you for your attention



FROM RESEARCH TO INDUSTRY



French Alternative Energies and Atomic Energy Commission

Commissariat à l'énergie atomique et aux énergies alternatives
Centre de Marcoule | 30207 Bagnols-sur Cèze Cedex

Etablissement public à caractère industriel et commercial | R.C.S Paris B 775 685 019

Nuclear Energy Division
Research Department on Mining and
Fuel Recycling Processes

Recycling the minor actinides, a potential contribution for decreasing the waste burden

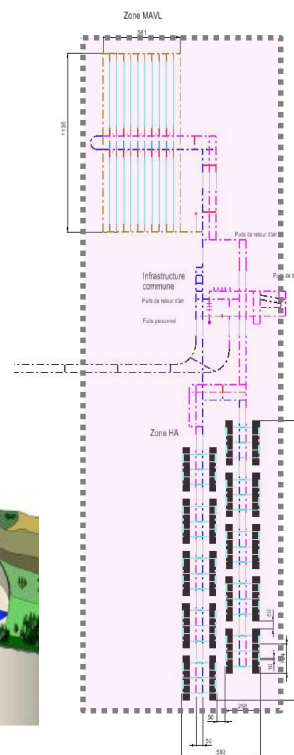
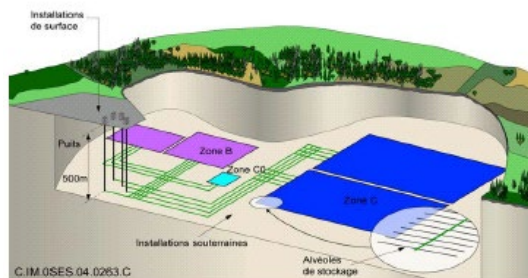
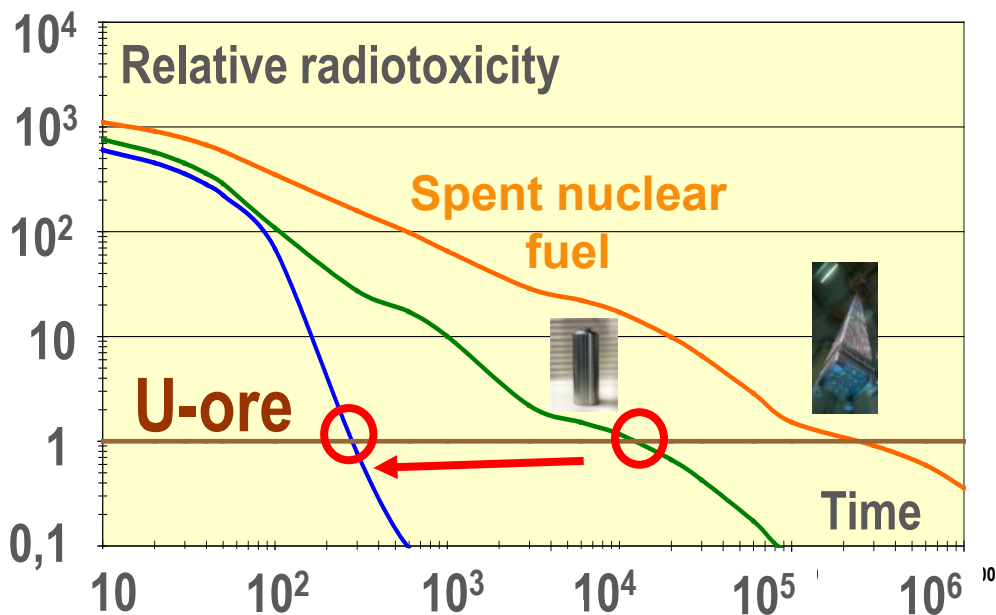
➤ Recycling MA \Leftrightarrow decrease waste lifetime and toxicity

➤ Recycling MA \Leftrightarrow preserve the "repository resource"

■ \searrow of the heat load $\Leftrightarrow \nearrow$ density of the repository

■ Interest in the sole-Am recycling

■ Strong \nearrow of repository "lifespan"



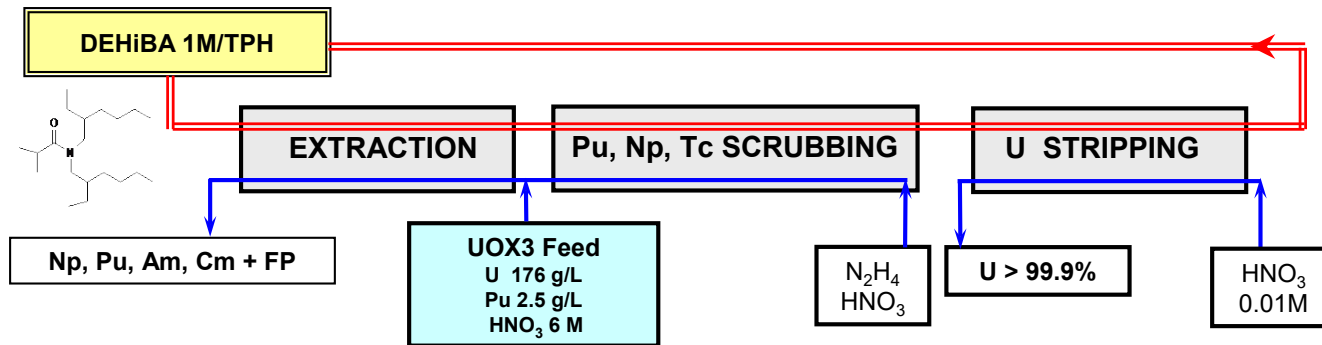
Am recycling

HLW: 160 ha

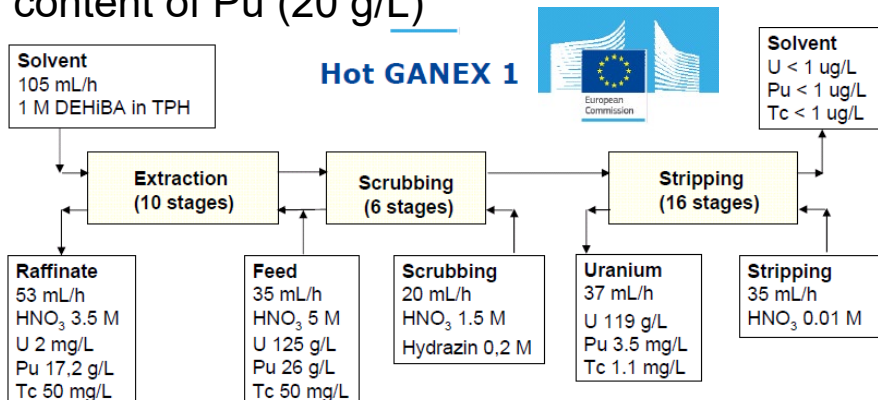
HLW: 1200 ha

GANEX 1 HOT TESTS

- HA test first performed at CEA (ATALANTE, CBP) in 2008 on genuine spent fuel in 28 mixer-settlers
 - Uranium recovery >99.99% with high DF (Np, Pu, Tc, Ru, Cs)



- HA test performed at ITU in 2012 within ACSEPT contract in 16 centrifugal contactors with high content of Pu (20 g/L)

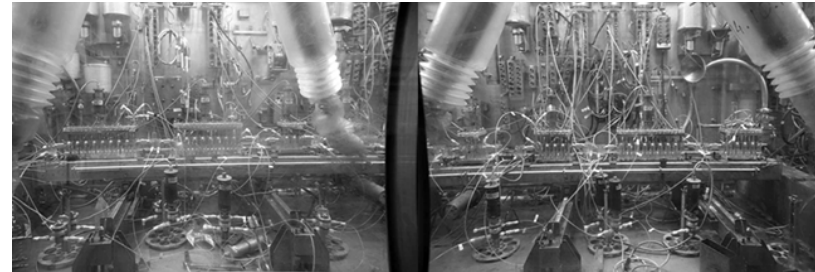


mg/L	Feed	Raffinate	Loaded organic	U fraction	Organic out
U	101300	10	36500	48000	21
Pu	20200	18300	9.2	24	0
Am	320	240	0	0	0
Cm	Below Detection Limit				
Np	17	13	0.25	0.4	0
Tc	41	18	3.2	3.5	1.6

Np almost completely routed with the raffinate
Small Tc losses to the U fraction

➤ 1st EXAm hot test on a HAC (dec-2015)

- Good hydrodynamic behaviour
- Needs an accurate process monitoring
- Compromise between Am recovery and Am/Cm decontamination factor



Performances :

- **2.4 g of Am recovered**
- **DF (Am/Ln) \approx 2000**
- **DF (Am/Cm) \approx 50**
- **Loss of Am in the raffinate \sim 10 %**
- **Loss of Am in the Mo output $<$ 0.01 %**

Element	Purity (%)
Am	95,5
Cm	0,6
Y	0,0
La	0,0
Ce	0,2
Pr	0,1
Nd	1,4
Sm	0,0
Eu	0,0
Fe	0,3
Mo	0,1
Zr	0,0
Pd	0,7
Ru	1,1

- Am very well recovered from fission products but decontamination towards Cm lower than expected
 - Due to a inaccurate modelling of Cm extraction in presence of TEDGA in these conditions
 - 40 stages of Cm scrubbing would have been required to reach 98% Am recovery and DF (Am/Cm) \approx 500