

# Long-term behaviour of extraction systems for nuclear fuel recycling

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*Topical day*

*When partitioning meets transmutation*

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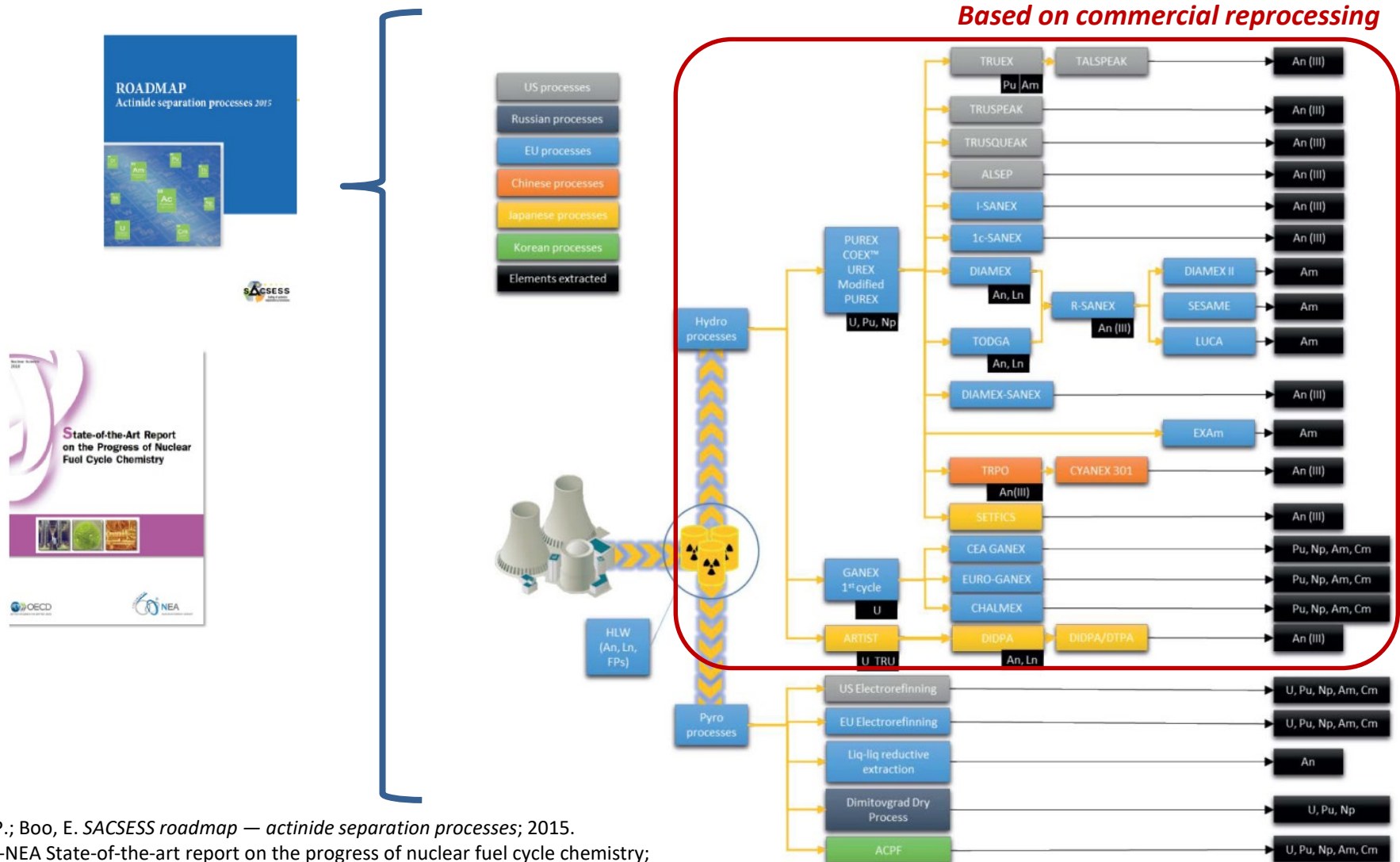
# Outlines

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1. *Introduction*
  - ❑ *Extraction process development*
2. *Degradation of solvents and long-term behavior*
3. *Stability studies*
4. *Stability studies along process development*
5. *Main conclusions*

# 1. Introduction

## SEPARATION PROCESSES for nuclear fuel recycling

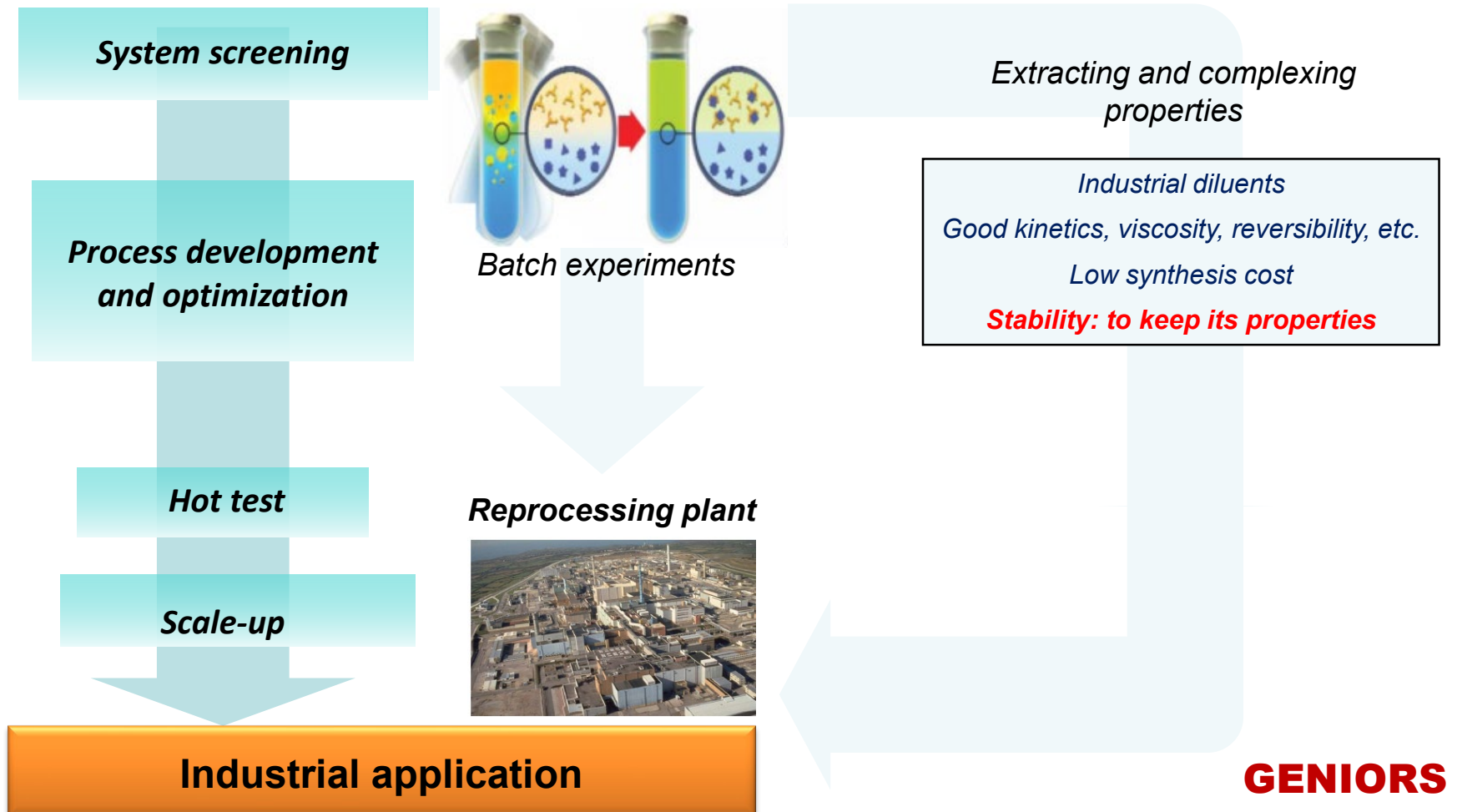


Joly, P.; Boo, E. SACSESS roadmap — actinide separation processes; 2015.  
 OECD-NEA State-of-the-art report on the progress of nuclear fuel cycle chemistry;  
 NEA No. 7267, OECD, Nuclear Energy Agency (NEA), Paris: 2018.

# 1. Introduction

## Extraction process development for nuclear fuel recycling

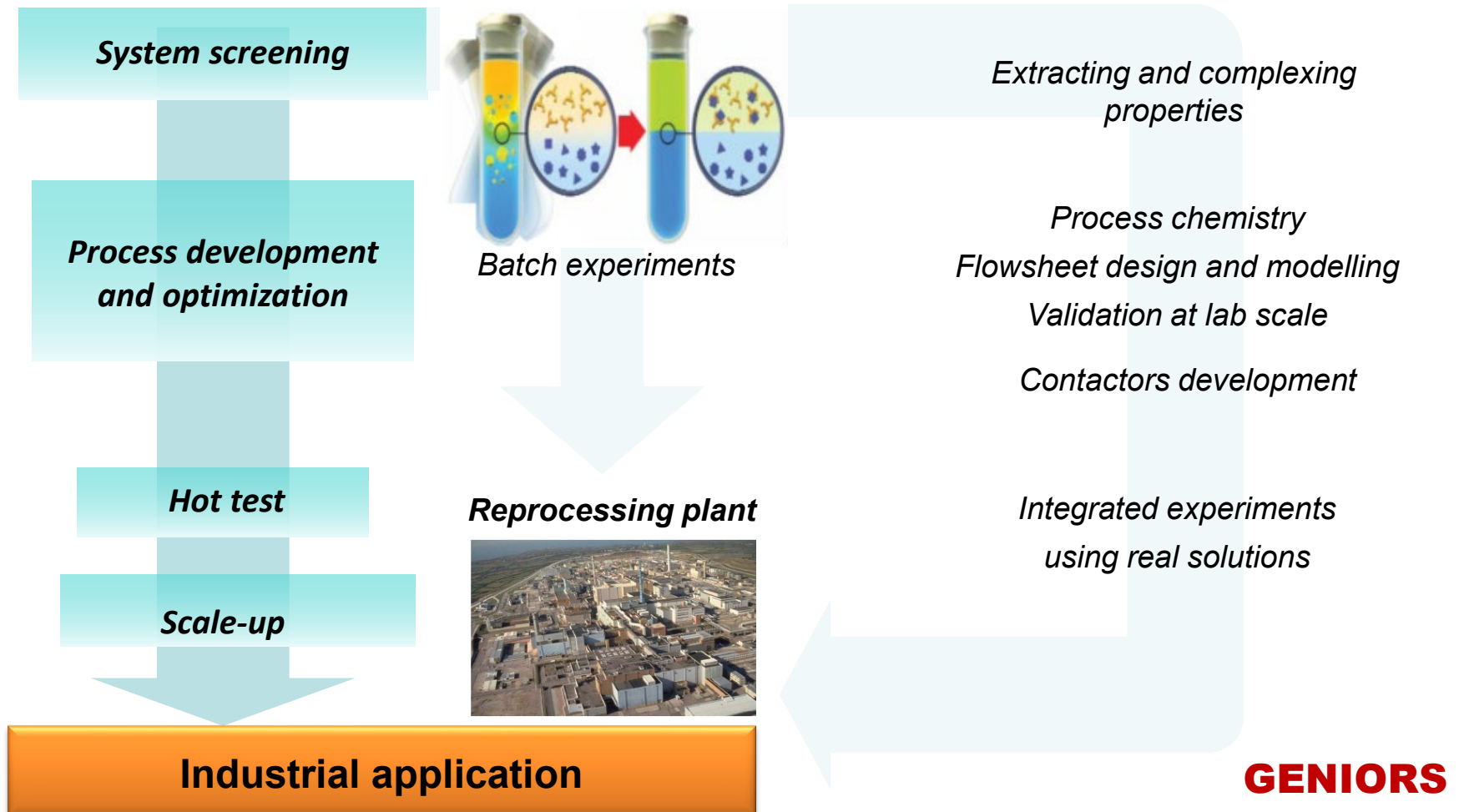
### Steps and limiting points



# 1. Introduction

## *Extraction process development for nuclear fuel recycling*

### Steps and limiting points



## 2. Degradation of solvents and long term behaviour

*What happens to a solvent during the process operation*

Evaporation or carry-over  
effects to other phases

Thermal degradation

Chemical attacks

Hydrolytic degradation

Radiolytic degradation

### *Stability studies*

*Avoid loss efficiency*

*Identification of any  
unexpected behavior*

*Reduce costs*

### *Important changes*

☹ Changes in the composition



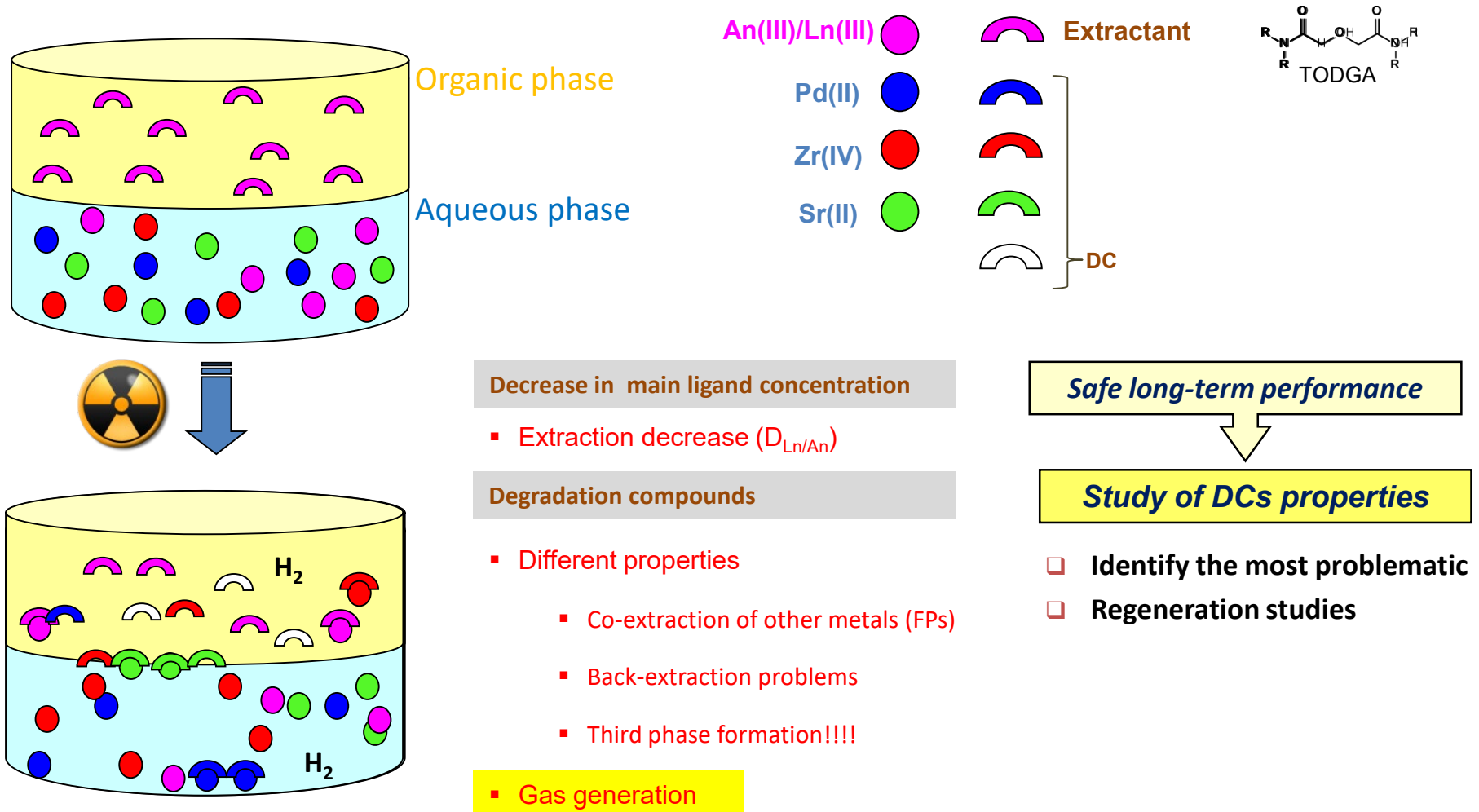
☹ Changes in physico-chemical  
and chemical properties

☹ Increase of secondary waste

☹ Increase of costs

## 2. Degradation of solvents and long term behaviour

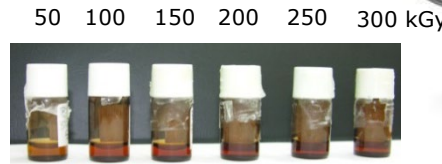
### Changes in the composition: main ligand and degradation compounds



### 3. Stability studies

*To simulate and study the effects to understand and predict*

**Simulate irradiation**



**How long?**

**What problems  
I have?**

**Why?**



*Modifying experimental conditions  
To understand stability rules!!!  
To find solutions!!!!*



**Improve systems**

**Regeneration  
of systems**



**Identify  
operative limits**

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### 3. Stability studies

#### *Different approaches to simulate effect of nuclear fuel radiation*

##### Type of radiation:

- ☐ ALPHA radiation (in-situ radiation)
- ☐ He ion beam
- ☐ GAMMA radiation ( $^{60}\text{Co}$  or  $^{137}\text{Cs}$ )

##### Design of experiments:

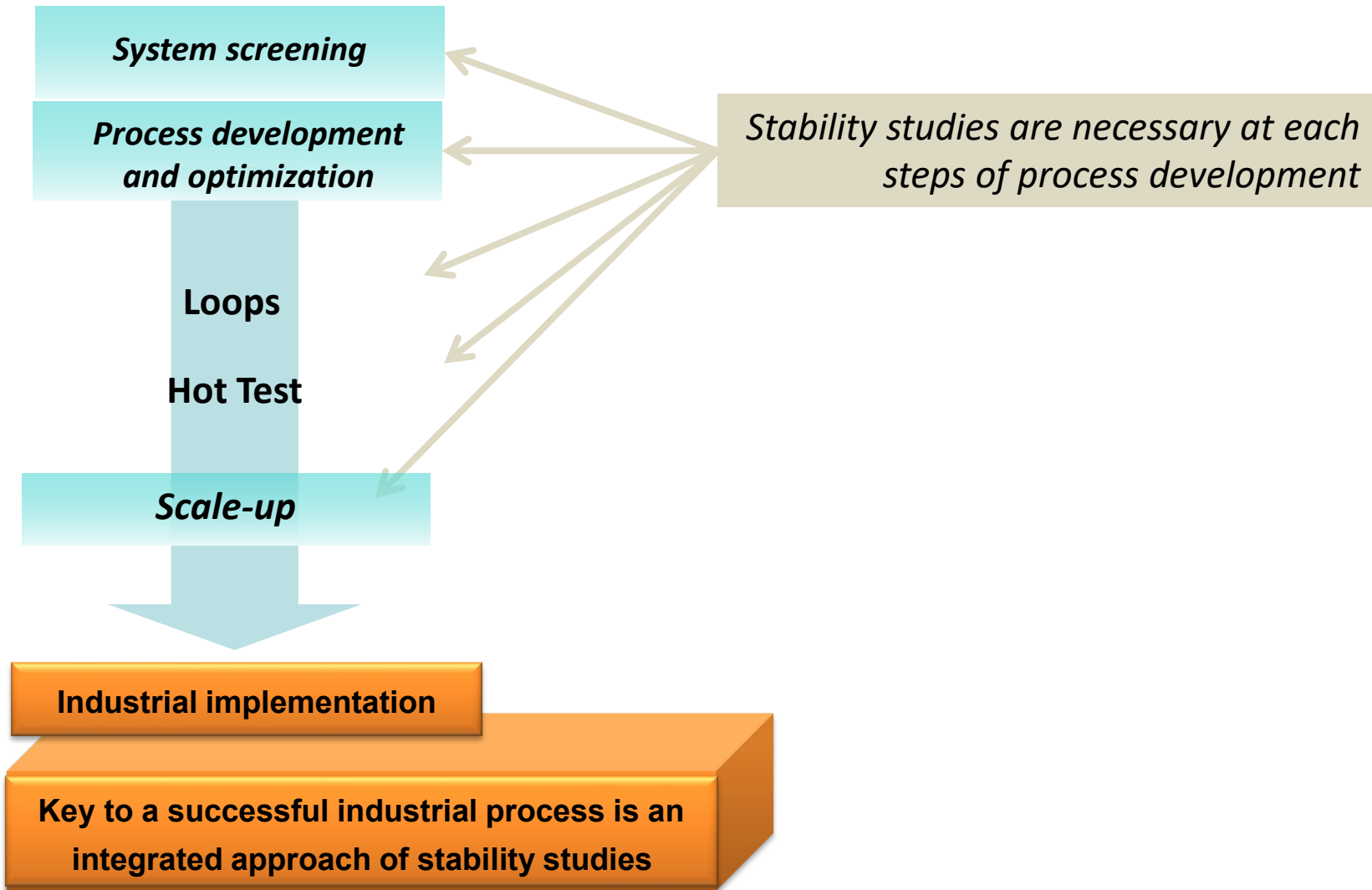
- Dose rate and integrated dose
- Static (batch) irradiation experiments
  - One or two phases in contact
- Dynamic (loops) irradiation experiments

##### Facilities available in GENIORS consortium

- ☐ SCK•CEN, Belgium
- ☐ Chalmers, Sweden
- ☐ Manchester (Dalton Cumbria), UK
- ☐ NNL, UK
- ☐ Náyade, CIEMAT, Spain
- ☐ INL, US, (GENIORS-DOE collaboration)
- ☐ Marcel, CEA, France

## 4. *Stability studies a long process development*

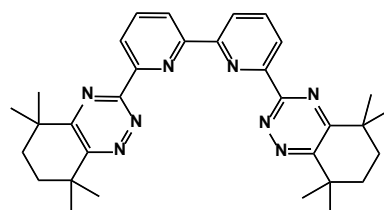
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## 4.1 Stability studies a long process development: Batch experiments I

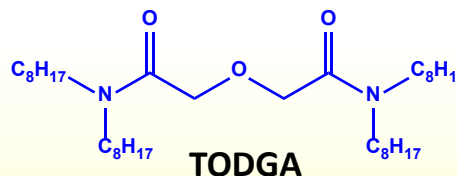
### System screening

- Extraction decrease ( $D_M$ ) as function of dose
- Decrease of main extractant concentration as function of dose



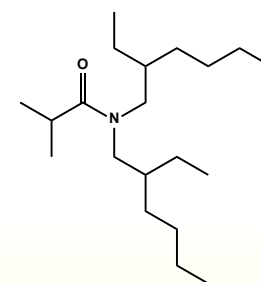
CyMeBTBP

Medium/low stability  
(~ 100-300 kGy)



TODGA

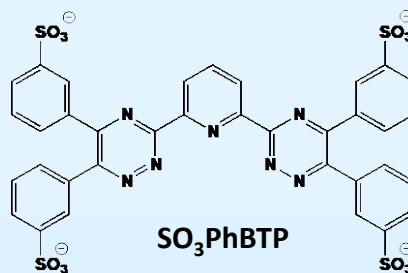
High stability  
(~ 500 kGy)



DEHIBA

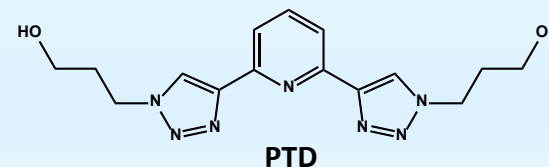
Very high stability  
(> 500 kGy)

Hydrophobic



SO<sub>3</sub>PhBTP

50% [L]<sub>0</sub> ~ 60 kGy



PTD

$D_M$  unaffected up to 200 kGy  
50% [L]<sub>0</sub> ~ 100 kGy

Water soluble

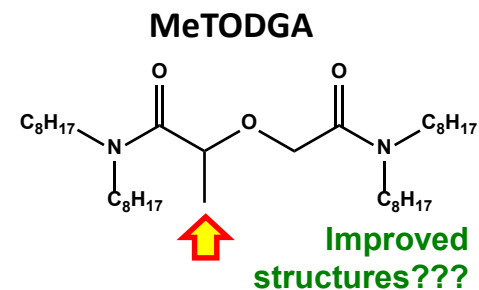
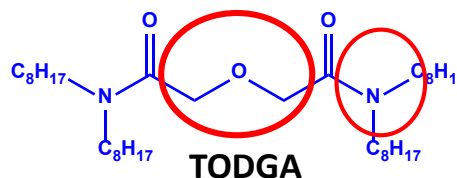
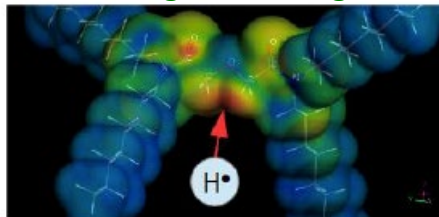
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## 4.1 Stability studies a long process development: Batch experiments I

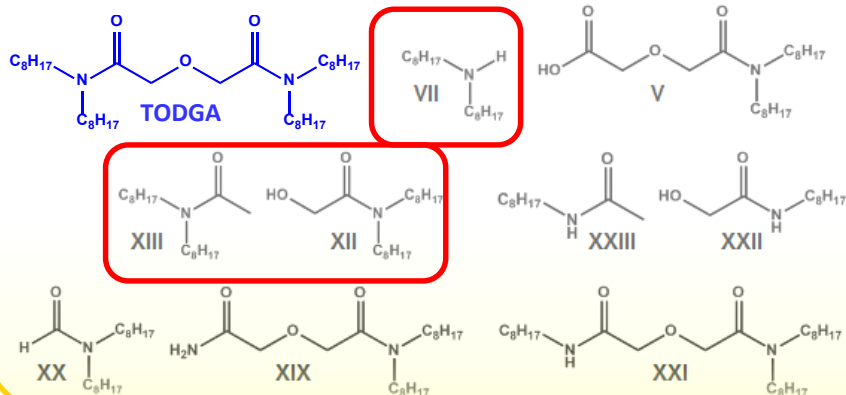
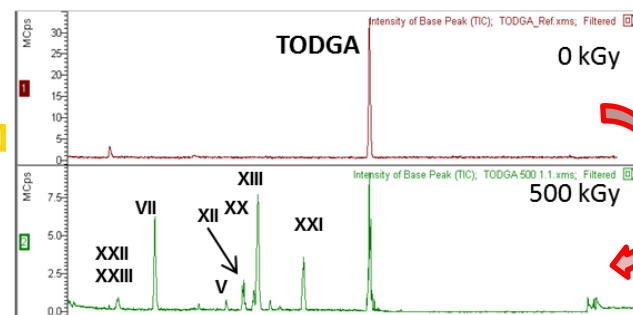
### System screening

- Extraction decrease ( $D_M$ ) as function of dose
- Decrease of main extractant concentration as function of dose
- Identification of degradation compounds**
- Weakest point of the molecule**
- Effects of diluents, pre-treatment and phase modifier

Modelling studies agree!!



### HPLC-MS spectra



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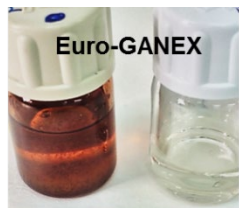
## 4.1 Stability studies a long process development: Batch experiments II

### System screening

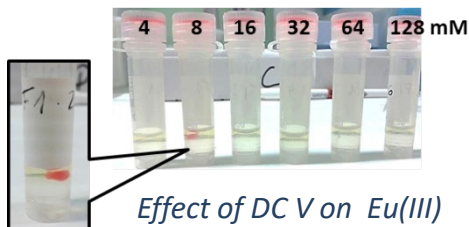
### Process development and optimization

- Effects on the performance
- Phase transferences or losses
- Possible accumulation of DCs

- Co-extraction or back extraction problems
- 3<sup>o</sup> phase, kinetics, loading capacity
- Phase Disengagement Time Ratio (DTR)
- Density, viscosity and hydrodynamic

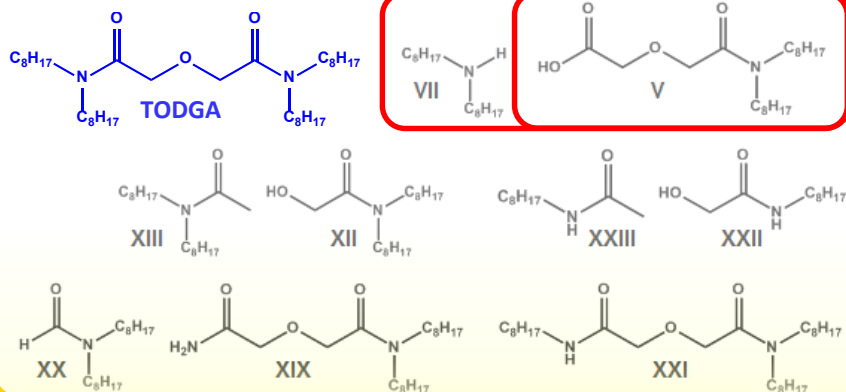


3 DCs responsible of insolubilities observed.



Effect of DC V on Eu(III) loading capacity

### DCs properties



### Accumulation:

More back-extraction stages would be necessary for Ln recovering

## 4.1 Stability studies a long process development: Batch experiments II

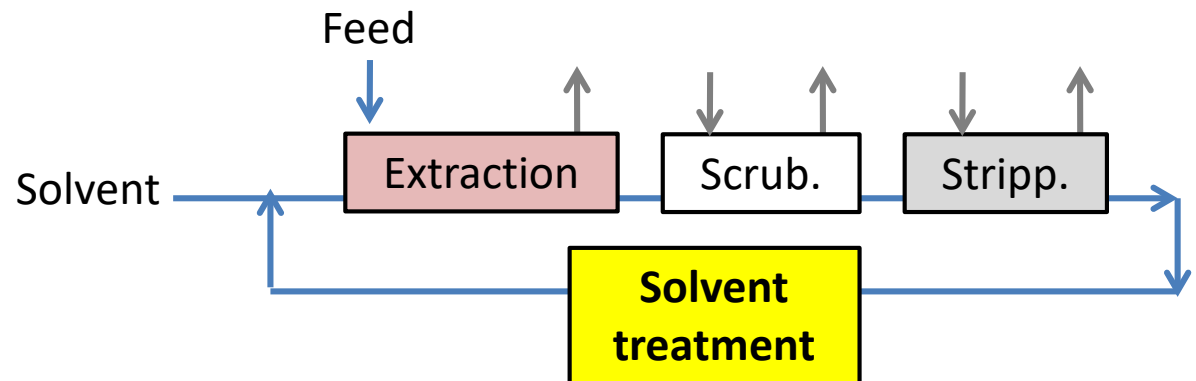
### System screening

### Process development and optimization

- Effects on the performance
- Phase transferences or losses
- Possible accumulation of DCs
- **Optimization of the flowsheets**

- Co-extraction or back extraction problems
- 3<sup>o</sup> phase, Kinetics, loading capacity
- Phase Disengagement Time Ratio (DTR)
- Density, viscosity and hydrodynamic

- Additional extraction, scrubbing or stripping steps
- **Additional steps for clean-up**



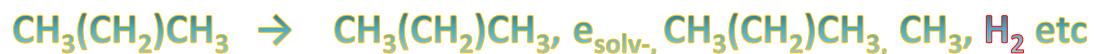
- Solvent treatment
- ❖ Basic washing
  - ❖ Acidic washing

## 4.2 Stability studies a long process development: Safety first!

**System screening**

**Process development  
and optimization**

**Gas generation:  $H_2$  production measurements**



To understand its production:

- Diluents effect
- Nitric acid effect
- Phase modifier effects

**U. Manchester, NNL and U. Lancaster collaboration:  
TODGA-based solvents**

**$He^{2+}$  Irradiations**

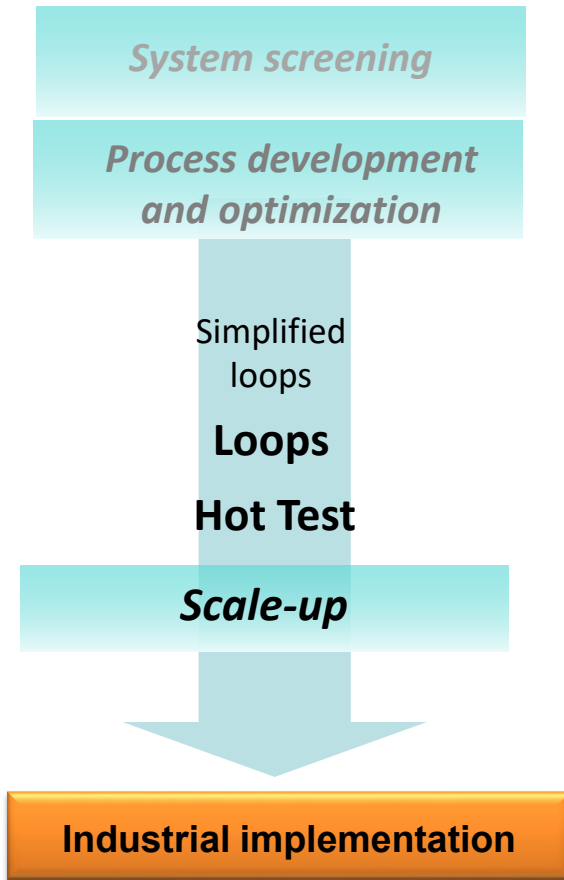


**Gamma irradiation in static vessels**



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## 4.3 Stability studies a long process development: Dynamic experiments



### *Continuous flowsheet implementations*

#### **Dynamic experiments**

- Control of extractant concentration and adjust solvent supplies
- Monitor accumulation of products and their impact
- Long term behavior of the solvent (recycling and treatment)



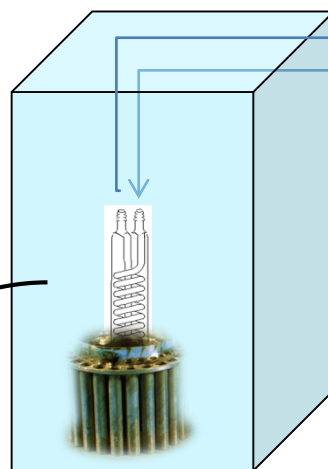


### 4.3.1 Dynamic experiments: Simplified IRRADIATION LOOPS

Dynamic experiments,  $\gamma$  Náyade irradiation facility, CIEMAT (Spain)

- Continuous irradiation ( $^{60}\text{Co}$  sources)
- Analysis
  - Solvent extraction properties
    - Distribution ratios
    - Ligand concentration
    - Degradation products
    - Acid concentration

Náyade pool



Extraction

Clean-up



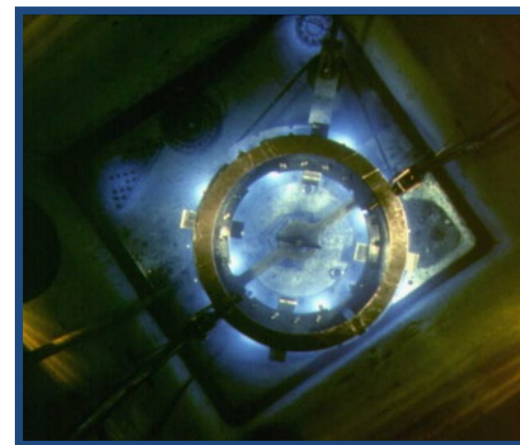
Irradiation device



Glass contactors



Glass coil

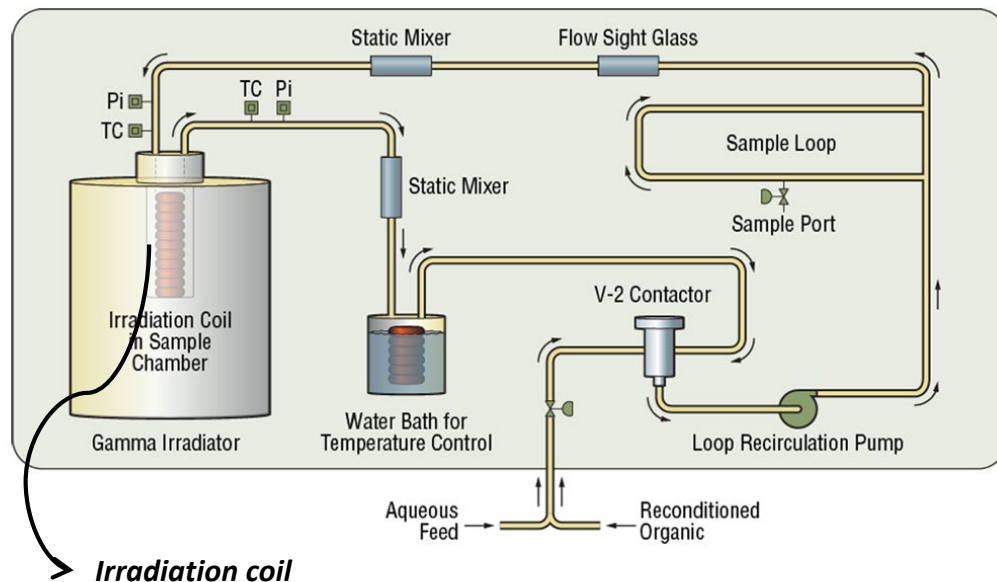


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## 4.3.2 Dynamic experiments: IRRADIATION LOOPS I

### Irradiation loop, INL (US)

- Continuous irradiation ( $^{60}\text{Co}$  sources)
- Recirculation under process-like conditions of irradiated mixed phases
- Analysis
  - Solvent extraction properties
    - Distribution ratios
    - Ligand concentration
    - Degradation products
    - Acid concentration
  - Phase disengagement times



**Irradiation coil**



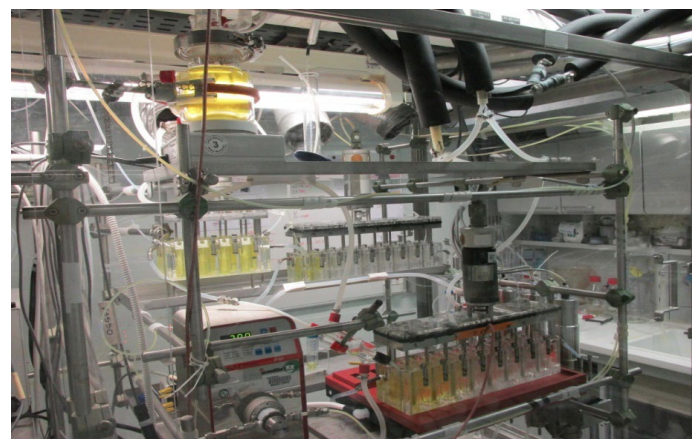
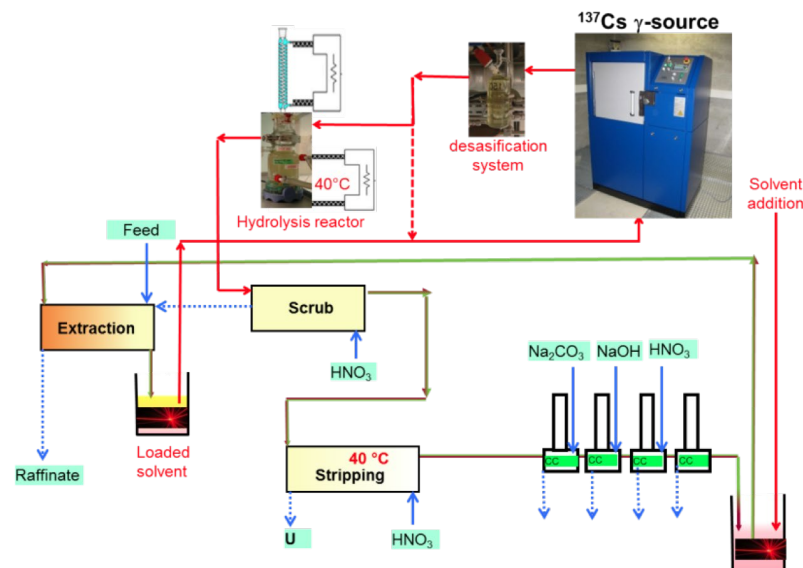
**Reconditioning loop**



## 4.3.2 Dynamic experiments: IRRADIATION LOOPS II

### MARCEL $\gamma$ Irradiation facility, CEA (France): A process platform

- Continuous irradiation ( $^{137}\text{Cs}$  sources)
- Continuous flowsheet implementations
  - Recycling and treatment of solvent
  - Control of extractant concentration and adjust solvent supplies
  - Monitor of breakdown accumulation products and impact on solvent properties
- Analysis
  - Distribution ratios
  - Ligand concentration
  - Degradation products
  - Physico-chemical properties
  - **Gas generation**



## 5. Conclusions

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*Development of an extraction process for nuclear fuel recycling*



*Solvent degradation must be understood to control normal and mal-operation*

### **Studies of long-term behavior must be an integrated approach**

- ❖ Stability of the molecules
- ❖ Identification of losses of efficiency
- ❖ Degradation products and their impact
- ❖ Identification of risks, limits and mal-operation situations
- ❖ Identification of recycling steps and evaluation of costs



*Batch experiments*



*Irradiation loop platforms*

# Acknowledgment

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## *All GENIORS partners*

<b>CEA</b>	<b>JRC-ITU</b>	<b>UEDIN</b>
<b>CHALMERS</b>	<b>JUELICH</b>	<b>UNIMAN</b>
<b>CIEMAT</b>	<b>KIT</b>	<b>UNIPR</b>
<b>CNRS</b>	<b>LGI</b>	<b>ULEEDS</b>
<b>CTU</b>	<b>NNL</b>	<b>UREAD</b>
<b>ICHTJ</b>	<b>POLIMI</b>	<b>ULANC</b>
<b>IIC</b>	<b>SCK-CEN</b>	<b>EDF</b>
<b>IRSN</b>	<b>TWENTE</b>	<b>AREVA</b>



## *Cooperation agreement with*



**SYTRAD I**  
**SYTRAD II**



***When partitioning meets transmutation***



2<sup>nd</sup> half year meeting

***Thank you for your attention***

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